

Research Article

Coincidence Summing Factor Calculation for Volumetric γ -ray Sources Using Geant4 Simulation

Dalal Abdullah Aloraini,¹ Mohamed Elsafi ,² Aljawhara H. Almuqrin,¹ and M.I. Sayyed³

¹Department of Physics, College of Science, Princess Nourah Bint Abdulrahman University, P.O.Box 84428, Riyadh 11671, Saudi Arabia

²Physics Department, Faculty of Science, Alexandria University, Alexandria 21511, Egypt

³Department of Physics, Faculty of Science, Isra University, Amman 11622, Jordan

Correspondence should be addressed to Mohamed Elsafi; mohamedelsafi68@gmail.com

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Geant4 simulation was applied to correct the coincidence summing (CS) effect in detecting a volumetric γ -ray sources, and this technique was applied to a ^{152}Eu standard sources. The radioactive sources were a liquid cylindrical, rectangular, and Marinelli beaker shapes of different volume for each one. Radionuclide track (RT) including coincidence summing and monoenergetic track without coincidence summing. The results obtained from two approaches compared with the experimental data and the modified KORSUM code for cylindrical γ -ray source. The comparison showed that the adopted method in this investigation is useful for coincidence summing corrections for a voluminous γ -ray sources. Moreover, this technique requires far less computation time than the techniques that depend on the calculation of total efficiency.

1. Introduction

To measure and analyze radioactive materials we need a high-resolution detector which is HPG, but when it is used to detect photons, the possibility of detecting more than one photon at the same time is possible especially with a multi-energy source, thus the CS effect is involved. The effect of CS usually loses peak count, so, corrected counts are needed to correct due to the CS effect. This problem exists when the distance between the radioactive sample (a source or environmental sample or nuclear materials that emit multiline radiation) and the detector is small. Also, if the size of the detector is large, the absorption of photons will be higher and therefore the CS effect will be present. ICRM was the first to report this effect in the 1980s [1].

The correction factor (CF) of the CS depends on the calculation of peak-total efficiency or direct total efficiency and that is discussed by many authors by different methods and techniques such as total efficiency calculation using the

Monte Carlo code [2], matrix form equations to calculate the total efficiency [3, 4], the GESPECOR code [5, 6], the code KORSUM and modifiedKORSUM code [7, 8], and the EFFTRAN code using the efficiency transfer technique or ETNA program [9, 10]. Total efficiency calculations sometimes become more difficult, especially for volumetric radioactive sources, where the changing of source or detector shape needed different complicated equations lead to less accuracy and long time for calculations, so the current technique determine the CS correction factor for different geometric sources using two options of GS without total efficiency calculation.

Kajimoto et al. calculated the CS correction for ^{24}N point source in case of close detector-to-source geometry using EGS5 code Moto Carlo simulation [11]. Yucel et al. used a semiempirical formula to calculate the true CS corrections based on the total efficiency calculation and this method can be applied without any difficulty to Ge detectors for coincident nuclide [12]. Taibi et al. used the MCNP5 code

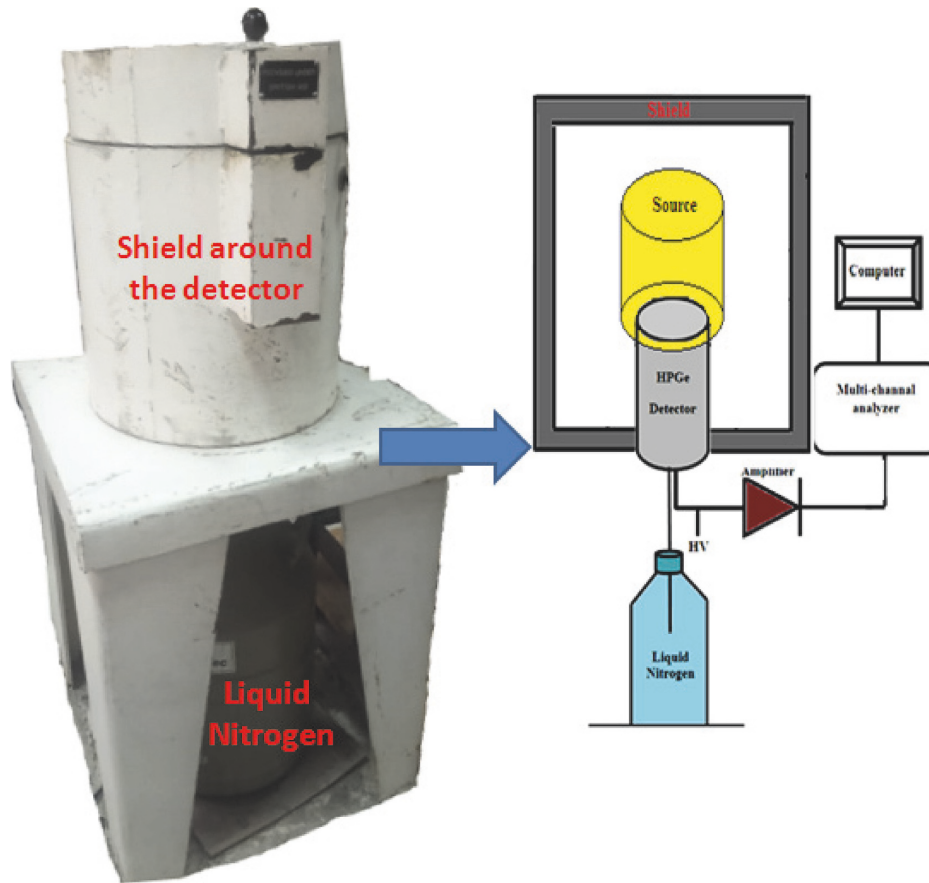


FIGURE 1: Illustration of the experimental setup.

Monto Carlo simulation to evaluate the true CS corrections for volumetric Eu-152 sources in gamma-ray spectroscopy, the results were confirmed with the TrueCoinc software, and a good agreement was obtained [13]. The problem of true CS correction for each of the point and volumetric sources was investigated through many previous works especially with the use of environmental sources or samples with low activity [14–25].

This work aims to utilize the Geant4 simulation (GS) to evaluate the correction factor for different volumetric ^{152}Eu sources (cylindrical and Marinelli beaker) using two options: monoenergetic and radionuclide tracks. In monoenergetic track (MT), the primary events have a monoenergetic energy while in a radionuclide track (RT), the complete decay scheme must be taken in the simulation and the primary events have multilines based on the decay scheme of radioactive sources.

2. Experimental Work

In this work, a coaxial HPGe spectrometry (Figure 1) has been used; its volume approximately was 110 cm^3 with a wide range of energy from 40 kV to 10 MeV for γ -ray detection. The detector dimensions are tabulated in Table 1. As mentioned in

TABLE 1: Main technical characteristics of HPGe detector provided by the company.

Crystal diameter	48 mm
Crystal length	54.5 mm
End cap distance	5 mm
Entrance window	1.5 mm
Dead layer	0.5 mm
The detector relative efficiency	24%
The resolution (FWHM) at 1.33 MeV	1.95 keV

the radioactive sources used as shown in Figure 2, different volumetric sources with different shapes filled with radioactive liquid, the activity of them referred to 15 Sep 1989 are tabulated in Table 2. The detector efficiency and energy was calibrated by using radioactive two point sources (Cs-137, Co-60, and Eu-152) before the measurement.

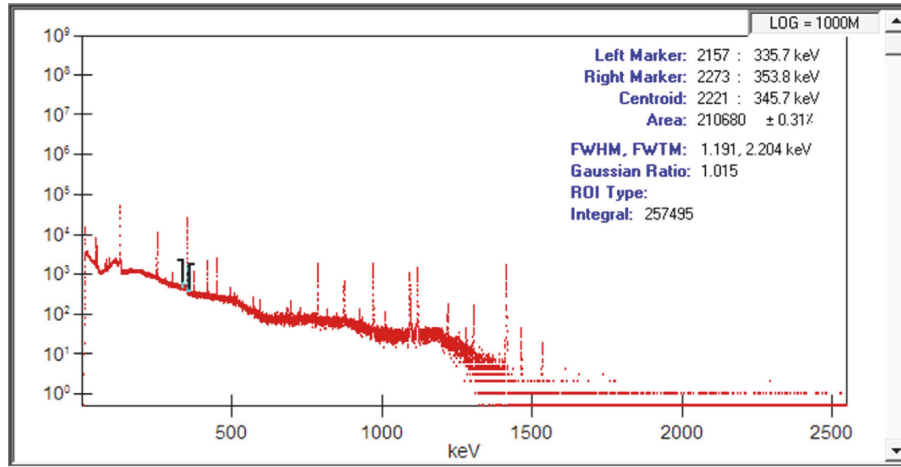
The measured time was too long to obtain high and sufficient counts under each peak (see Figure 3). The analysis of the spectrum is performed by the region of interest (ROI) selecting. This is defined by ROI or the area of interest. The determination of ROI by choosing the start and end channels of the peak as required. The analysis using Genie-2000 software provides integrated count calculation as well



FIGURE 2: The radioactive volumetric sources used in the measurements.

TABLE 2: The characteristics of different volumetric sources used in practical measurements.

Items	Source description			
	Cylindrical sources		Marinelli beaker sources	
Code	C1	C2	M1	M2
Volume (mL)	400	500	800	1000
Height (mm)	41.69	52.25		Of well = 40
Radius (mm)		56.94		Maximum = 56.8
Wall thickness (mm)		2.03	2.7	Minimum = 38.5
Jar material			Polypropylene	
Activity (kBq)at 19 Sep 1989			48.1 ± 6	


 FIGURE 3: The experimental spectrum using ^{152}Eu volumetric sources in log-scale measured by HPGe detector.

as the background count calculations depend on ROI. In addition, the Genie-2000 software has a big facility to carry out automatic search as well as peak area calculations using the changing in peak fitting using an interactive fit-peak.

The uncorrected measured photopeak efficiency of a certain energy (E) was evaluated by the following equation:

$$\varepsilon_e(E) = \frac{N_e(E)/t}{P_\gamma(E)Ae^{-\lambda T}}, \quad (1)$$

where, $N_e(E)$ is the net measured counts of full-energy peak after subtracted by the background at energy E , t is the actual time for experimental measurement, $P_\gamma(E)$ is the probability of photon emission at a given energy E , A is the standard

radionuclide activity in Bq, $e^{-\lambda T}$ represent the decay factor, where T is the production time and λ represent the decay constant. The uncertainty in the full-energy peak (FEP) efficiency is denoted by σ_ε , and calculated by the following expression:

$$\sigma_\varepsilon = \varepsilon \cdot \sqrt{\left(\frac{\partial \varepsilon}{\partial A}\right)^2 \cdot \sigma_A^2 + \left(\frac{\partial \varepsilon}{\partial P}\right)^2 \cdot \sigma_P^2 + \left(\frac{\partial \varepsilon}{\partial N}\right)^2 \cdot \sigma_N^2}, \quad (2)$$

where, σ_N , σ_P , and σ_A , are the associated uncertainties with the net count rate N , emission probability P , and activity A , respectively. The corrected measured FEP efficiency is calculated by the following expression:

$$\varepsilon_t = \varepsilon_c \cdot CF, \quad (3)$$

where CF is the correction factor of the CS effect.

3. Geant4 Simulation (GS)

GS is a process of simulated radiation (photons, neutrons, protons, or electrons) and interact during materials. These simulations include multiple functions, including physical processes, geometry, analysis, and tracking. Physical processes contain multiple functions, including electromagnetic operations, interactions of radiation, and analyzes [17]. The simulation of the detector and the source is made in a large volume of the “world volume” as shown in Figure 4. Figure 4 represents the simulation of HPGe detector and cylindrical source and collimator. In this work, the simulated radiation was γ -ray photons with a radionuclide, including decay scheme (multiple energies) or using monoenergetic option (monoenergetic photons).

3.1. Radionuclide Track (RT). The decay scheme of radionuclides represents the primary events, where the decay scheme of Eu-152 source was simulated in details, including all energies emitted from this source. These energies were interacted with the HPGe crystal and the results of all interaction types (photoelectric, compton, and pair production) registered in a histogram file. After simulating the program, the run occurred with a number of events at least 10^7 events and the generated spectrum as shown in Figure 5 was obtained using root software. Gaussian distribution was used to fit each peak in the spectrum to calculate the area under this peak. The simulated FEP efficiency using the radionuclide track at certain energy, denoted by ε_1 and its equation is as follows:

$$\varepsilon_1 = \frac{\text{Area under the peak}}{\text{Primary events} \cdot I_\gamma}. \quad (4)$$

3.2. Track 2: Monoenergetic Track (MT). Monoenergetic gamma rays represent the primary events, where a single energy from Eu-152 source was simulated in details. These events with the sample energy were interacted with the HPGe crystal and the results of all interaction types (photoelectric, compton, and pair production) registered in a histogram file. After simulating the program, the run occurred with a number of events at least 10^7 events and the generated spectrum as shown in Figure 6 was obtained using root software. Gaussian distribution was used to fit each peak in the spectrum to calculate the area under this peak. To limit the error within 1%, a number of particles (primary events) 10^7 are used in each simulation, otherwise the error has well gotten higher as the number goes less [15, 16]. The simulated FEP efficiency using the monoenergetic track at certain energy, denoted by ε_2 and its equation is as follows:

$$\varepsilon_2 = \frac{\text{Area under the peak}}{\text{Primary events}}. \quad (5)$$

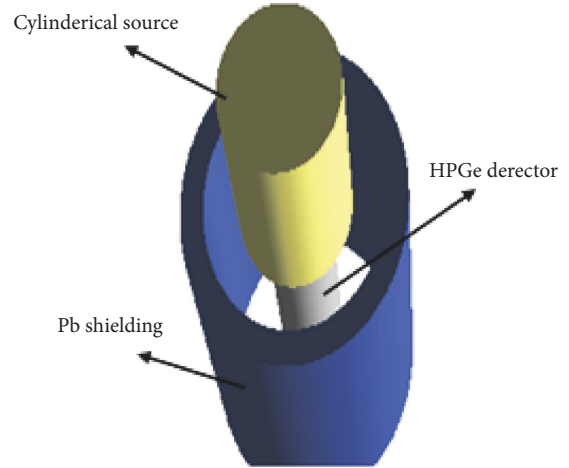


FIGURE 4: A model of radioactive cylindrical source and HPGe detector shielded by lead.

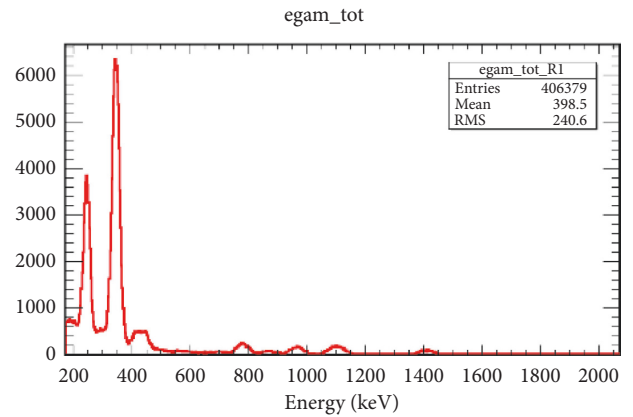


FIGURE 5: The simulated spectrum in using radionuclide decay shows the number of counts at different γ -ray energies.

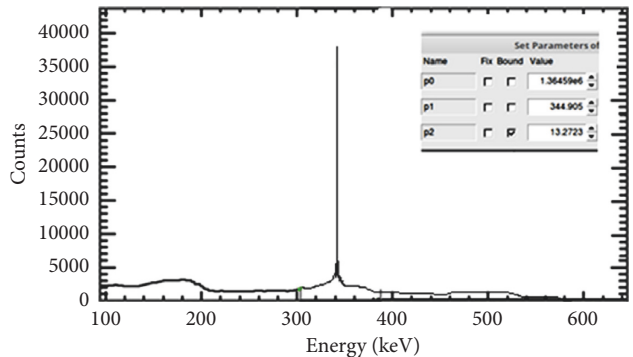
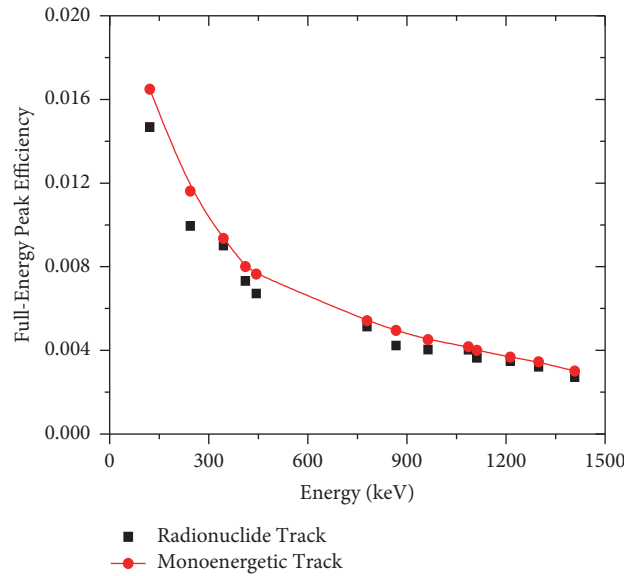


FIGURE 6: The simulated spectrum at 344 keV using Gaussian distribution in monoenergetic track.

3.3. CF Calculation. The correction factor of the CS can be estimated from the two options, where the radionuclide track contains multiple lines and is detected at the same time, so the possibility of coincidence summing exists, and in the second track (MT), just a monoenergetic line was detected or simulated so there is no coincidence summing.

TABLE 3: The calculated photopeak area and its uncertainty for two different tracks as well as the full-energy peak efficiency for both the options.

Energy (keV)	RT		MT		ϵ_1	ϵ_2
	Area	Uncertainty	Area	Uncertainty		
121.78	366907	5.98E + 02	412000	7.17E + 02	0.014676	0.01648
244.7	248693	4.24E + 02	290250	6.76E + 02	0.009948	0.01161
344.28	225366	3.91E + 02	233750	5.98E + 02	0.009015	0.00935
411.12	182760	3.51E + 02	200250	4.24E + 02	0.00731	0.00801
443.97	167838	3.17E + 02	191000	3.91E + 02	0.006714	0.00764
778.91	128199	3.01E + 02	135250	3.51E + 02	0.005128	0.00541
867.38	105589	2.89E + 02	123750	3.17E + 02	0.004224	0.00495
964.082	101021	6.71E + 02	112750	3.01E + 02	0.004041	0.00451
1085.84	100618	7.07E + 02	104250	2.89E + 02	0.004025	0.00417
1112.08	91091	7.17E + 02	100000	6.71E + 02	0.003644	0.004
1212.95	87196	6.76E + 02	92000	7.07E + 02	0.003488	0.00368
1299.15	80188	5.98E + 02	86250	7.17E + 02	0.003208	0.00345
1408.01	67732	4.24E + 02	75250	7.17E + 02	0.002709	0.00301

FIGURE 7: Comparison of ^{152}Eu full-energy peak efficiency calibration curve using two different tracks using cylindrical source of 1000 mL put at the top of the detector.

The correction factor calculated by the following equation is as follows:

$$CF = \frac{\epsilon_2}{\epsilon_1}. \quad (6)$$

4. Results and Discussion

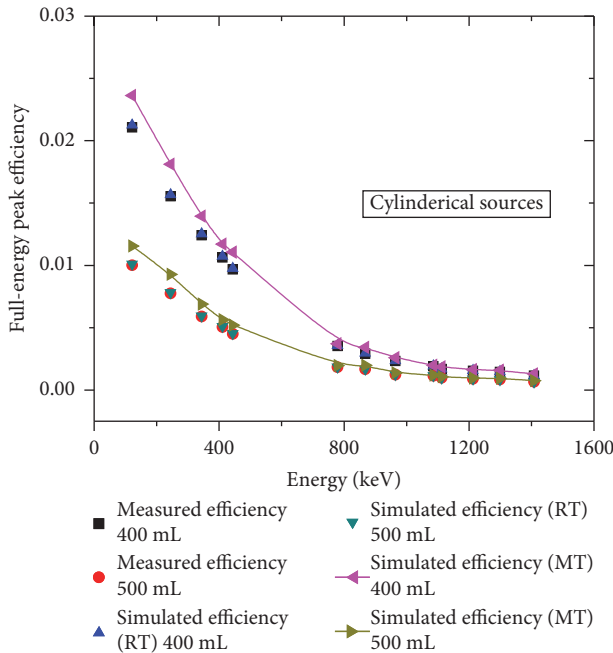
First, the detector geometry and the radioactive source in [8] are modeled using the Geant4 code, where the detector crystal dimensions were 58.2 mm diameter and 79 mm length. The cylindrical source activity was 86.2 ± 1.5 kBq (date: 2013 May 1). The source is distributed uniformly inside a cylinder (100 mm ϕ , 149 mmH) filled with 0.1 M HCl

and has an active volume of 1000 ml. The FEP efficiency was calculated by radionuclide and monoenergetic tracks, the net area under the peak was calculated and tabulated in Table 3, from this area and the number of primary events, the simulated FEP efficiency was calculated as shown in Figure 7. The FEP efficiency by using MT higher than the FEP efficiency by RT et al. discussed energies, where at 244.7 keV from 1000 ml cylindrical source, the FEP efficiency in by using MT was 0.01161, and the FEP efficiency by using RT was 0.009948, while at 1408.1 keV, the FEP efficiencies were 0.00301 and 0.00271, respectively.

The CF was calculated using equation 6 at different energies of the cylindrical ^{152}Eu radioactive source. The calculation of CF in the present simulated technique was

TABLE 4: Comparison between the present work and the modified KORSUM code and the deviation error between them.

Decay mode	Energy (keV)	Correction factor (CF)		$\Delta\%$
		Modified KORSUM [8]	Present work Geant4	
$EC(\beta^+)$	121.78	1.1235	1.1229	-0.05
	244.7	1.1674	1.1671	-0.03
	443.97	1.1375	1.138	0.04
	867.38	1.17	1.172	0.17
	964.082	1.1154	1.1161	0.06
	1085.84	1.036	1.0361	0.01
	1112.08	1.0977	1.0978	0.01
	1212.95	1.0553	1.0551	-0.02
β^-	1408.01	1.101	1.111	0.90
	344.28	1.0377	1.0372	-0.05
	411.12	1.0961	1.0957	-0.04
	778.91	1.0551	1.055	-0.01
	1299.15	1.0756	1.0757	0.01

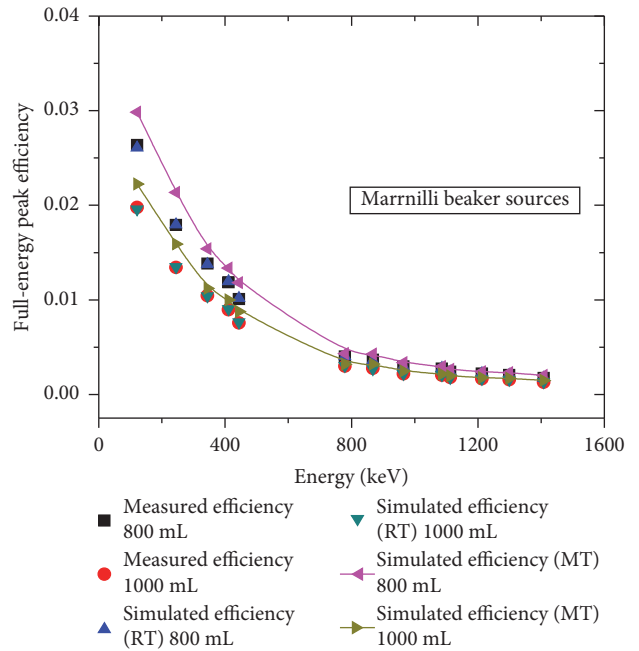
FIGURE 8: The experimental, simulated RT, and simulated MT full-energy peak efficiency for different cylindrical volumetric ^{152}Eu sources as a function of different γ -ray energies as a function of different γ -ray energies.

compared with the modified KORSUM code [8], and the relative deviation ($\Delta\%$) was calculated by

$$\Delta\% = \frac{CF_{\text{Geant4}} - CF_{\text{KORSUM}}}{CF_{\text{Geant4}}} \cdot 100. \quad (7)$$

The results were tabulated in Table 4 and showed an excellent compatibility between the two methods. This indicated that this simulation technique using Geant4 is correct to calculate the CS corrections by simple and faster computations.

Secondly, the FEP efficiency was determined experimentally using cylindrical and Marinelli beaker radioactive sources with different volume according to section 2. This

FIGURE 9: The experimental, simulated RT, and simulated MT full-energy peak efficiency for different Marinelli beaker volumetric ^{152}Eu sources as a function of different γ -ray energies.

efficiency was compared with the simulated FEP efficiency by using monoenergetic and radionuclide tracks as shown in Figure 8 for cylindrical sources and Figure 9 for Marinelli beaker sources. The results indicated that the experimental and radionuclide simulated FEP efficiency have almost the values energy ranges, but the monoenergetic simulated FEP efficiency is higher, that is due to the existence of the CS in experimental and radionuclide simulated methods.

Finally, from the ratio between the monoenergetic and radionuclide simulated FEP efficiency, the correction factor of the coincidence summing effect for the HPGe detector using all present radioactive sources was calculated in Table 5. It is clear that the lower volume has the higher correction factor, for example, the correction factor for cylindrical source C1 = 1.143 (400 ml) and for C2 (500 ml) is 1.112, that

TABLE 5: The correction factor of the coincidence summing effect using different radioactive volumetric sources.

Energy	CF			
	C1	C2	M1	M2
121.78	1.143	1.112	1.143	1.136
244.7	1.188	1.156	1.188	1.181
344.28	1.160	1.116	1.119	1.078
411.12	1.104	1.091	1.117	1.106
443.97	1.147	1.133	1.160	1.147
778.91	1.063	1.042	1.071	1.061
867.38	1.171	1.158	1.188	1.189
964.082	1.115	1.103	1.131	1.133
1085.84	1.035	1.028	1.058	1.055
1112.08	1.086	1.089	1.119	1.108
1212.95	1.066	1.047	1.079	1.067
1299.15	1.097	1.077	1.107	1.091
1408.01	1.134	1.112	1.143	1.127

is due to the absorption of source container. If the volume increases, the self-absorption will increase, and then the correction factor will decrease.

5. Conclusion

The FEP efficiency of HPGe detector was calculated by radionuclide and monoenergetic tracks in Geant4 simulation using different shapes and volumes radioactive sources. The correction factor of CS was evaluated based on the two simulated efficiencies by fast and accurate way compared with evaluating the correction factor by total efficiency calculation. The FEP efficiency by using MT is higher than the FEP efficiency by RT et al. discussed energies, where at 244.7 keV from 1000 ml cylindrical source, the FEP efficiency in by using MT was 0.01161 and the FEP efficiency by using RT was 0.009948, while at 1408.1 keV, the FEP efficiencies were 0.00301 and 0.00271, respectively. The CF was calculated for cylindrical and Marinelli beaker sources with different volumes. The lower volume has the higher correction factor in the same shape of radioactive sources, that is due to the self-absorption of the volumetric source.

Data Availability

All data are available in the manuscript.

Conflicts of Interest

The authors declare that they have no conflicts of interest.

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